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Safety Evaluation

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Safety Evaluation Number ¹ : SE-W375-00-00013	Revision No: 0	
ABCN Number: ABCN-W375-00-00016	_	
Safety Evaluation Subject: Unrestricted Area Dose Rates		

PART I: DESCRIPTION OF THE PROPOSED REVISION, BACKGROUND, AND SCHEDULE

- 1. Describe the proposed revision (including credible failure modes, if applicable).
 - Clarifies statement in ISMP section 3.9.1.2 to note that ALARA features are shown on applicable facility layout drawings and other design documents not on one specific "set of radiation protection drawings."
 - Replaces reference to ISMP sections 2.3 and 3.91.2 as implementing standards in SRD Vol. II Safety Criterion 2.0-3 with G-10 CFR 835/B2, Occupational ALARA Program.
- 2. Identify the affected Authorization Basis (AB) documents and perform a comparison and assessment of the revision against the AB.
 - ISMP sections 2.3 and 3.9.1.2: These ISMP sections are currently cited in SRD Safety Criterion 2.0-3 as implementing standards; the citation will be replaced with G-10 CFR 835/B2. Also, ISMP section 3.9.1.2 is being revised to clarify the project's approach for documenting ALARA design features.
 - SRD Vol. II Safety Criterion 2.0-3: The implementing standard for this safety criterion will be changed from ISMP sections 2.3 and 3.9.1.2 to G-10 CFR 835/B2.

The assessment of the change against the AB is contained in section II below.

- 3. List the references used for the safety evaluation.
 - 1. BNFL-5193-ISP-01, Rev. 4b, Integrated Safety Management Plan
 - 2. BNFL-5193-SRD-01, Rev. 2e, Safety Requirements Document
 - 3. BNFL-TWP-SER-003, Rev. 3, River Protection Project Waste Treatment Plant Radiation Protection Program For Design, November 15, 1999
 - 4. DOE/RL-96-0006, Revision 1, July 1998, *Top-Level Radiological, Nuclear, and Process Safety Standards and Principles for TWRS Privatization Contractors*, U.S. Department of Energy, Richland Operations Office
 - 5. RL/REG-97-07, Revision 0, 6/25/97, *Guidance for the Review of TWRS Privatization Contractor Integrated Safety Management Plan Submittal Package*, June 1997, DOE/RL Office of Radiological, Nuclear, and Process Safety Regulation for TWRS Privatization Contractors, Richland, WA
 - 6. RL/REG-98-03, Revision 0, *DOE Regulatory Unit Evaluation Report of the BNFL Inc. Integrated Safety Management Plan*, March 1998, DOE/RL Office of Radiological, Nuclear, and Process Safety Regulation of TWRS Privatization Contractors, Richland, WA
 - 7. RL/REG-99-20, Revision 0, *Guidance for the Review of TWRS Privatization Contractor Revised Standards Approval Package for Construction Authorization*, August 31, 1999, DOE/RL Office of Radiological, Nuclear, and Process Safety Regulation of TWRS Privatization Contractors, Richland, WA
 - 8. G-10 CFR 835/B2, Rev. 1, Implementation Guide for Use with Title 10, Code of Federal Regulations, Part 835, Occupational Radiation Protection Occupational ALARA Program, DOE Assistant Secretary for

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¹ The Safety Evaluation Number shall be obtained from Project Document Control.

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	Environment, Safety and H	Health, November 1994			
4.	Describe the planned revision i	implementation schedule.			
	The changes to the SRD and IS	MP will be incorporated with	nin 30 days of RU approval.		
PA	RT II: REGULATORY IM	IPACT OF PROPOSED	AB REVISION		
	e following questions are to be answed the proposed initiating change if a			posed AB 1	revision
1.	Does the revision involve the deleti established in the approved SRD? JUSTIFICATION:	ion or modification of a star	ndard previously identified or	<u>YES</u>	<u>NO</u>
	SRD Vol. II Safety Criterion 2.0-3 implementing standards. These ISI Occupational ALARA Program.				
2.	Does the revision result in a reducti	ion in commitment currently	described in the AB?	\boxtimes	
	JUSTIFICATION:				
	The existing ISMP commits to havi document facility zoning, minimum Although it was never intended that drawings be developed, the language "set." The variety of ALARA design is such that they cannot all be depicted.	n shielding requirements, a nt a limited, defined set of '' ge in the ISMP could be con gn features such as those sp	nd access control features. Radiation Protection'' astrued to require such a ecifically listed in the ISMP		
	The revision to ISMP section 3.9.1. be shown on applicable facility layor commitment to document facility z control features remains unchanged	out drawings and other desi coning, minimum shielding	ign documents; the		
	The wording of SRD Vol. II Safety Inc. maintains its commitment to c 060 (1) and WAC 246-247-040 (2), any unrestricted area will not exceed	comply with the applicable in i.e., that maximum dose rate	portions of WAC 246-221- te from external sources in		
	G-10 CFR 835/B2 is more responsi replaces; therefore, the level of con and 3.9.1.2 do not address dose lim the subject of SRD Safety Criterion "External sources of radiation in a hours/year) shall be maintained be	nmitment is not reduced. Spaits from external sources in 2.0-3. Section I of G-10 83 reas of continuous occupati	pecifically, ISMP sections 2.3 unrestricted areas, which is 55/B2 requires that: ional occupancy (2,000		

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		YES	NO
	this average as is reasonably achievable (10 CFR 835.1002(b))" The continuous occupancy limit of 0.5 mrem/hr equates to an average dose rate of 0.0005 rem/hr, which is 25% of the limit in Safety Criterion 2.0-3 for unrestricted areas. While this standard does not specifically address dose rates in unrestricted areas, adherence to the guidance in G-10 835/B2 will ensure that the contribution to dose rates in unrestricted areas from radiation sources within the RPP-WTP is a fraction of the limit in Safety Criterion 2.0-3. In addition, the results of the most recent full-year (CY 98) readings from the ten TLDs which are in place around the perimeter of the construction site show that the average hourly dose rate from natural background radiation and the influence of other Hanford facilities is 0.010 mrem/hr, or 0.5% of the limit of Safety Criterion 2.0-3. It follows that offsite radiation sources will have a very small impact on dose rates within unrestricted areas of the RPP-WTP site. Therefore, by ensuring that dose rates in continuous occupancy areas of the plant are ALARA, the selected standard will help ensure that the limit of Safety Criterion 2.0-3 is not exceeded.		
	Furthermore, G-10 CFR 835/B2 provides design and operational guidance for maintaining radiation exposures ALARA; use of the guidance in this standard will help ensure that the limit of Safety Criterion 2.0-3 is not exceeded.		
3.	Does the revision result in a reduction in the effectiveness of any program, procedure, or plan described in the AB.		\boxtimes
	JUSTIFICATION:		
	The revision to ISMP section 3.9.1.2 merely clarifies that ALARA features will be shown on applicable facility layout drawings and other design documents. The effectiveness of the ALARA design program is not reduced, because the facility radiation zoning, minimum shielding requirements and access control features are still being provided in the design. G-10 CFR 835/B2 is more responsive to the SRD safety criterion than the ISMP sections it replaces; therefore, the effectiveness of the ALARA design program is not reduced.		
	te: Guidance on defining the terms and responding to the above questions is provided in K700 Managing Changes to the Authorization Basis, Appendix 6.	C528, Code o	of Practice
If a	all the answers to the above questions are no, then the change can be made without prior RU ap	proval.	
	any of the above answers is yes, then RU approval is required prior to implementation of the A tiating change if applicable). An ABAR shall be prepared to obtain RU approval (see K70C52		
PA	ART III: SAFETY EVALUATION CONCLUSION		
	All PART II questions are answered No. Therefore, RU approval is NOT required prior to proposed AB revision (and initiating change where applicable).	implementin	g the
\boxtimes	At least one PART II question is answered Yes. Therefore, RU approval IS required prior t proposed AB revision (and initiating change where applicable). Issuance of an ABAR is re		

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Safety Evaluation Subject: Unrestricted Area Dose Rates	
approval.	
Evaluator/Originator	Date
Reviewer ²	Date
Radiological, Nuclear, and Process Safety Manager	Date
Chair, Project Safety Committee ³	Date
RPP-WTP General Manager ³	Date

The reviewer should be a person from the same department as the Evaluator/Originator and at least as qualified as the Evaluator/Originator to conduct safety evaluations.
 This signature required if Safety Evaluation concludes AB change can be made without RU prior approval. If RU approval (ABAR) is required, PSC and GM signatures occur on the ABAR.